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DESIGN, SAFETY FEATURES AND PROGRESS OF THE HTR-PM

The high-temperature gas-cooled reactor pebble-bed module (HTR-PM) is aimed to extend nuclear energy application beyond the grid, including cogeneration, high-temperature heat utilization, and hydrogen production. The first concrete of HTR-PM demonstration power plant, which has been approved as part of the National Science and Technology Major Projects, was poured five years ago, in Rongcheng, Shandong Province, China. The thermal power of a single HTR-PM reactor module is 250 MWth, the helium temperatures at the reactor core inlet/outlet are 250/750 °C, and a steam of 13.25 MPa/567 °C is produced at the steam generator outlet. Two HTR-PM reactor modules are connected to a steam turbine to form a 210 MWe nuclear power plant. The progress of HTR-PM project in China has drawn considerable attention worldwide. In this webinar, the design basis, design principles, general design features and safety characteristics of HTR-PM will be presented. Main engineering verification experiments of components and systems for the HTR-PM, such as helium blower, steam generator, will be introduced. Progress of the HTR-PM demonstration power plant, including civil engineering, first-of-a-kind equipment manufacturing, licensing, installation of the main equipment, will be described. In addition, the irradiation test results of pebble fuel samples and the status of commercial fuel production plant will be explained.

Free webcast

Wednesday January 24, 2018 at 8:30 am EST (UTC-5)



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Who should attend: policy makers, managers, regulators, students, general public

Meet the Presenter...

Dr. Yujie Dong is a Professor in Nuclear Engineering at Tsinghua University, Beijing, China, where he earned his PhD degree in Nuclear Reactor Engineering and Safety. From 1997 he worked to develop advanced nuclear reactors at the Institute of Nuclear and New Energy Technology, INET, Tsinghua University. He was Head of the Division of Reactor Thermal-Hydraulic Calculation, Head of the Division of Reactor Physics, Thermal-hydraulics and system simulation. From 2006 he was responsible for the Division of General Design of High Temperature Gas-cooled Reactor (HTGR). Currently, he is the Deputy Director and Deputy Chief Engineer of INET in charge of HTGR projects. Also, he has been appointed by the National Energy Administration as Deputy Technical Director of the *HGTR Nuclear Power Plant Project*, which is one of the National Science and Technology Major Projects. He was actively involved in planning the System Arrangement of VHTR as a member of System Steering Committee in the frame of GIF.



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March 21 2018	SCK.CEN's R&D on MYRRHA, Prof. H.C Hamid Ait Abderrahim
April 25, 2018	Russia BN 600 and BN 800, Dr. Yuri Ashurko

For more information, please contact: Patricia Paviet at patricia.paviet@nuclear.energy.gov or visit the GIF website at www.gen-4.org

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