



INTRODUCING NEW PLANT SYSTEMS DESIGN (PSD) CODE

Prof. Nawal Prinja, Jacobs, UK
25 March 2021



Meet the Presenter



Professor Nawal Prinja, Technology Director of Jacobs (Clean Energy), has 40 years of academic and industrial experience in the nuclear sector. He holds a position of Honorary Professor at four British universities: School of Engineering, Aberdeen University; College of Engineering, Brunel University London; School of Engineering, Bolton University; and School of Computer Science and Electronic Engineering, the International Centre of Nuclear Engineering, Bangor University. Currently, he is working with WNA on Harmonization of Nuclear Codes. He has been on IAEA missions to China, South Africa, UAE, Spain and Poland. Dr. Prinja was appointed as an advisor to the UK Government to help formulate their long-term R&D strategy for nuclear industry and continues to advise as a member of the Fusion Advisory Board of UKRI and Nuclear Propulsion Science and Technology Advisory Group of Ministry of Defense. He participates in several international committees notably the ASME code committee for developing new Plant Systems Design code and represents the UK at the Senior Industry Advisory Panel of the Generation IV International Forum.



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- Need for Plant Systems Design (PSD) Standard
- Solution
- Objectives
- Related initiatives
- ASME PSD Committee – Structure and Charter
- Integrating safety and design

Current Position

- Technology Director, Jacobs
- Honorary Professor in the School of Engineering at Aberdeen University.
- Honorary Professor in the College of Engineering, Brunel University London.
- Honorary Professor, School of Engineering, Bolton University.
- Honorary Professor, School of Computer Science and Electronic Engineering, the International Centre of Nuclear Engineering, Bangor University.

Experience

- 40 years of engineering and technology experience in aerospace, automotive, oil & gas and nuclear power.
- Over 50 Technical publications including 3 books.

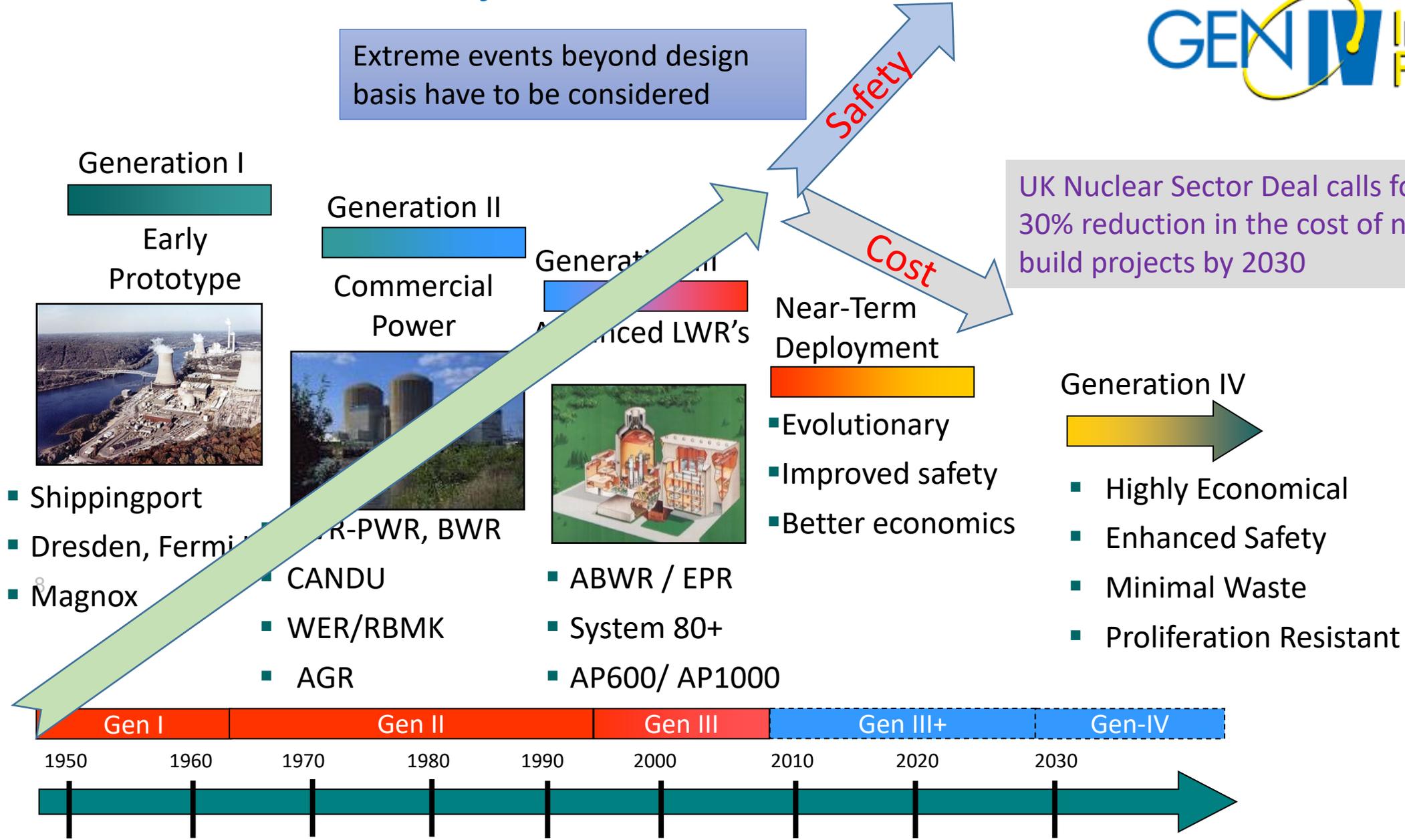
Appointments

- Advisor to the Ministry of Defence (MOD) on the Nuclear Propulsion Research & Technology programme for the nuclear submarines.
- Member of the Nuclear R&D Advisory Board to the UK Government.
- Chairman of WNA/CORDEL international Task Force for harmonisation of mechanical Codes & Standards and Vice-chair of CORDEL Steering Committee.
- Technical Expert invited by the IAEA (United Nations) to chair expert meetings on safety classification and Technology Readiness Levels and to participate in Nuclear Knowledge Management and Seismic expert missions to UAE, S Africa, China, Spain and Poland.
- Independent assessor appointed by the Innovate UK of UKRI.
- Member of the EC funded FENET and EASIT2 projects aimed at developing computer based simulation competencies.
- Ex-Member of Technical Assessment Panel of Fusion for Energy (F4E)
- Member of the Board of Directors for the Professional Simulation Engineer (PSE) certification scheme.
- Chair of Industry Advisory Committee for the National Structural Integrity Research Centre at Cambridge.
- Member of the Fusion Advisory Board, EPSRC of UKRI.
- UK representative at the Senior Industry Advisory Panel of Gen IV International Forum (GIF)
- Member of Plant Systems Design code committee of ASME.

Need to Increase Safety and Decrease Cost

Extreme events beyond design basis have to be considered

UK Nuclear Sector Deal calls for 30% reduction in the cost of new build projects by 2030



Generation I

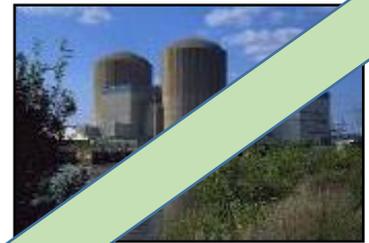
Early Prototype



- Shippingport
- Dresden, Fermi
- Magnox

Generation II

Commercial Power



- R-PWR, BWR
- CANDU
- WER/RBMK
- AGR

Generation III

Advanced LWR's

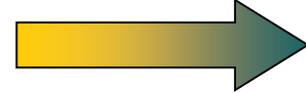


- ABWR / EPR
- System 80+
- AP600/ AP1000

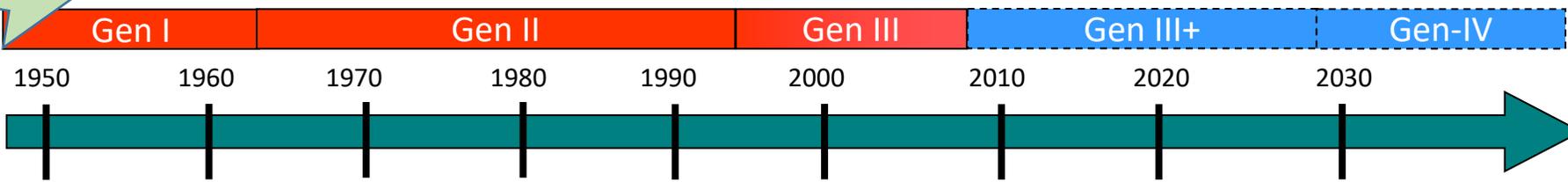
Near-Term Deployment

- Evolutionary
- Improved safety
- Better economics

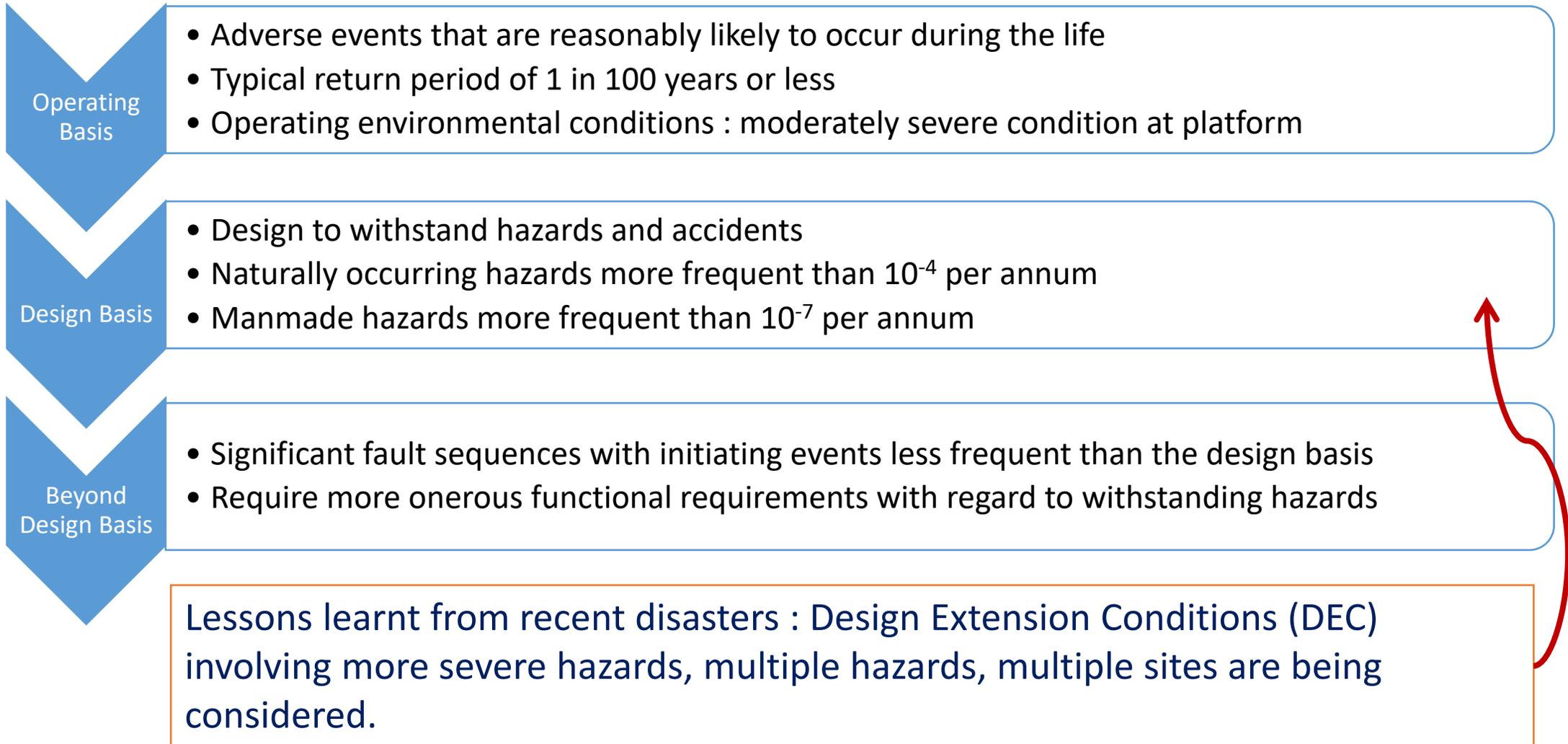
Generation IV



- Highly Economical
- Enhanced Safety
- Minimal Waste
- Proliferation Resistant



Extreme Events Beyond Design Basis : Nuclear Industry Approach



The Solution : New PSD Code



- The focus of this standard is to reduce the cost of new build and increase safety. It provides a technology independent framework, including requirements and guidance, for organizations implementing this Standard to:
- conduct plant process hazard evaluations and analyses in the early stages of plant design that
 - advance as the design matures
 - provide structure to the initial development of a quantitative risk assessment,
- integrate systems engineering design processes, practices and methods with traditional design processes, practices and tools, and
- integrate risk informed probabilistic design processes with traditional deterministic design methods and processes using reliability and availability targets.

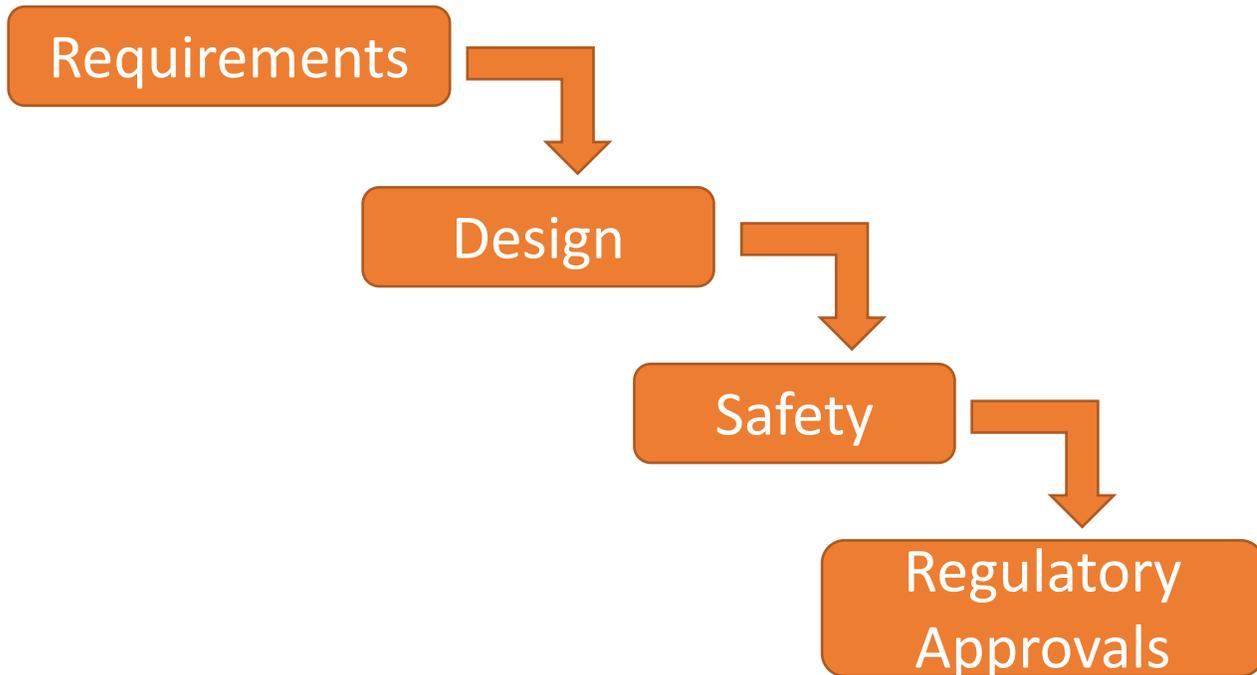
Systems Engineering



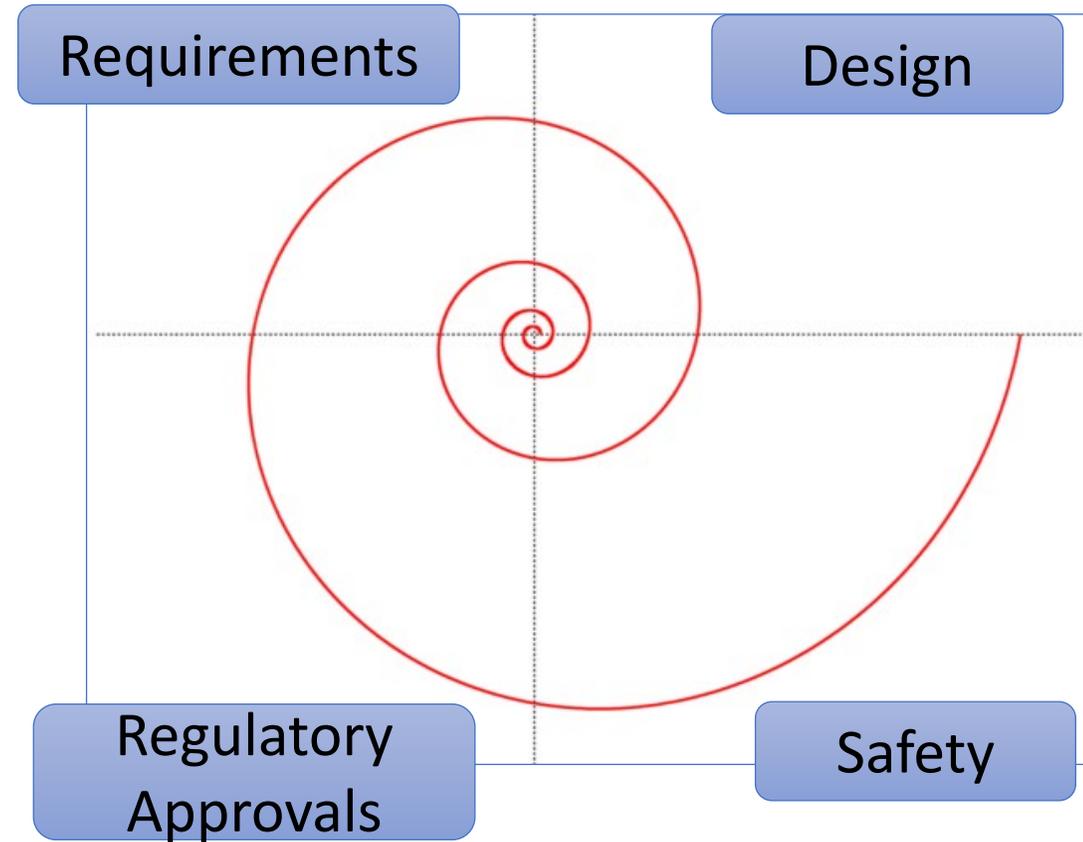
... a transdisciplinary and integrative approach, to enable the successful realization, use, and retirement of engineered systems, using systems principles and concepts, and scientific, technological, and management methods. (www.incose.org)

From a Waterfall to a Spiral

Current: Sequential Activity



Future: Integrating Safety & Design



The Objectives

(1 of 2)



1. **Safer** and **more efficient** system designs and design alternatives with **quantified safety levels**
2. **More effective requirements management**
3. More **cost-effective** and **timely** strategies for issue resolution and design maturation
 - e.g., alternatives analysis, design modifications, earlier formulation of safety function design criteria, additional research, laboratory testing, and scale testing

The Objectives

(2 of 2)



4. **Combine risk informed probabilistic design methodologies with traditional deterministic design methods** using reliability and availability targets developed by JCNRM* in accordance with a companion standard for Establishing Plant System & Component Reliability Targets.
5. Cover design of facility plant systems over the **entire life cycle** of a plant (design, construction, operation, decontamination and decommissioning)
6. Be **system based**, vs. component based, and cover multiple disciplines (mechanical, electrical, instrumentation & control, HVAC, etc.)

* ANS/ASME Joint Committee on Nuclear Risk Management (JCNRM)

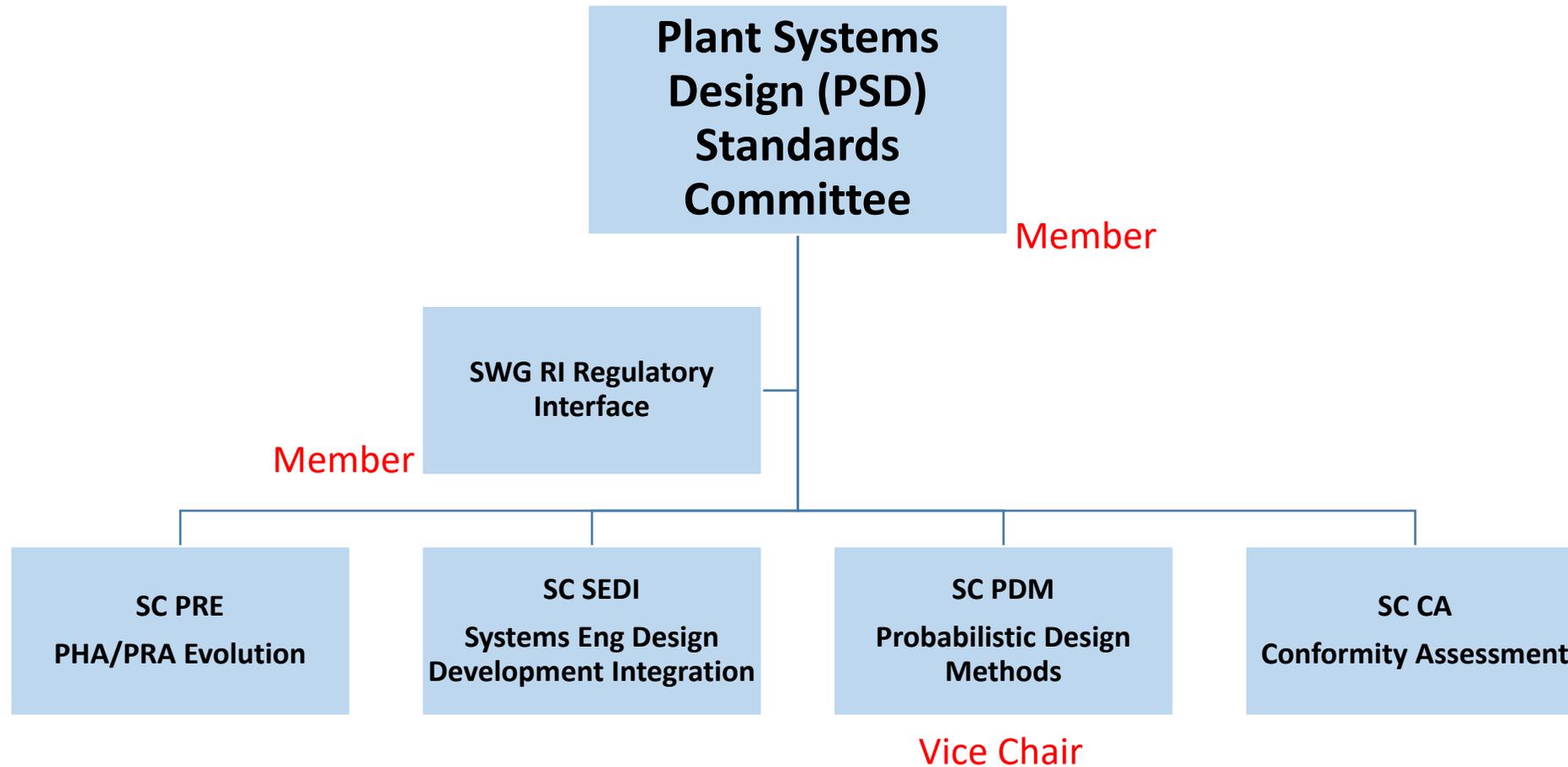
Related Initiatives



The following are ongoing and include similar objectives.

- ANS & ANS 30.1
- EPRI Body of Knowledge (BoK)
- ASME Section XI, Div. 2, Requirements for Reliability and Integrity Management Programs for Nuclear Power Plants (RIM)

Proposed PSD Committee Structure



Charter: PSD Subcommittee on PHA & PRA Evolution (SC PRE)



To develop, review and maintain technology neutral processes and procedures for design organizations to conduct **process hazard analysis** for nuclear, fossil, petrochemical, chemical, and hazardous waste plants and facilities **that can; (a) be integrated in the early stages of design, (b) advance as the design matures, and (c) provide structure to the initial development of a probabilistic risk assessment for advanced technologies and designs.** The focus is to provide requirements and guidance for hazard analysis and probabilistic risk assessment processes, methodologies and tools that will provide safer and more efficient system and component designs with quantified reliability levels.

Charter: PSD Subcommittee on Systems Engineering Design Development Integration (SC SEDI)



To develop, review and maintain technology neutral processes and procedures for design organizations to incorporate and integrate existing systems engineering processes, practices and tools with traditional architect engineer design development processes, practices and tools for design of nuclear, fossil, petrochemical, chemical, and hazardous waste plants and facilities. This includes integration with enabling technical management processes and design development tools and databases. The focus is to provide requirements and guidance for system development and design integration processes, methodologies and tools that will provide safer and more efficient system and component integrated designs with quantified reliability levels.

Charter: PSD Subcommittee on Probabilistic Design Methods (SC PDM)



To develop, review and maintain technology neutral processes and procedures for design organizations to **incorporate risk informed probabilistic design methodologies with traditional deterministic design methods using reliability and availability targets** for design of nuclear, fossil, petrochemical, chemical, and hazardous waste plants and facilities. The focus is to provide requirements and guidance for probabilistic design methodologies and tools that will provide safer and more efficient system and component designs with quantified reliability levels.

Risk Informed Performance Based (RIPB) Approach



RI

A risk-informed approach to decision-making represents a philosophy whereby risk insights are considered together with other factors to establish requirements that focus attention on design and operational issues commensurate with their importance to the health, safety, and the environment of the public.

PB

A performance-based approach is one that establishes performance and results as the primary basis for safety decision-making, and incorporates measurable (or calculable) parameters, objective criteria to assess performance are established based on risk insights, deterministic analyses and/or performance history.

RIPB

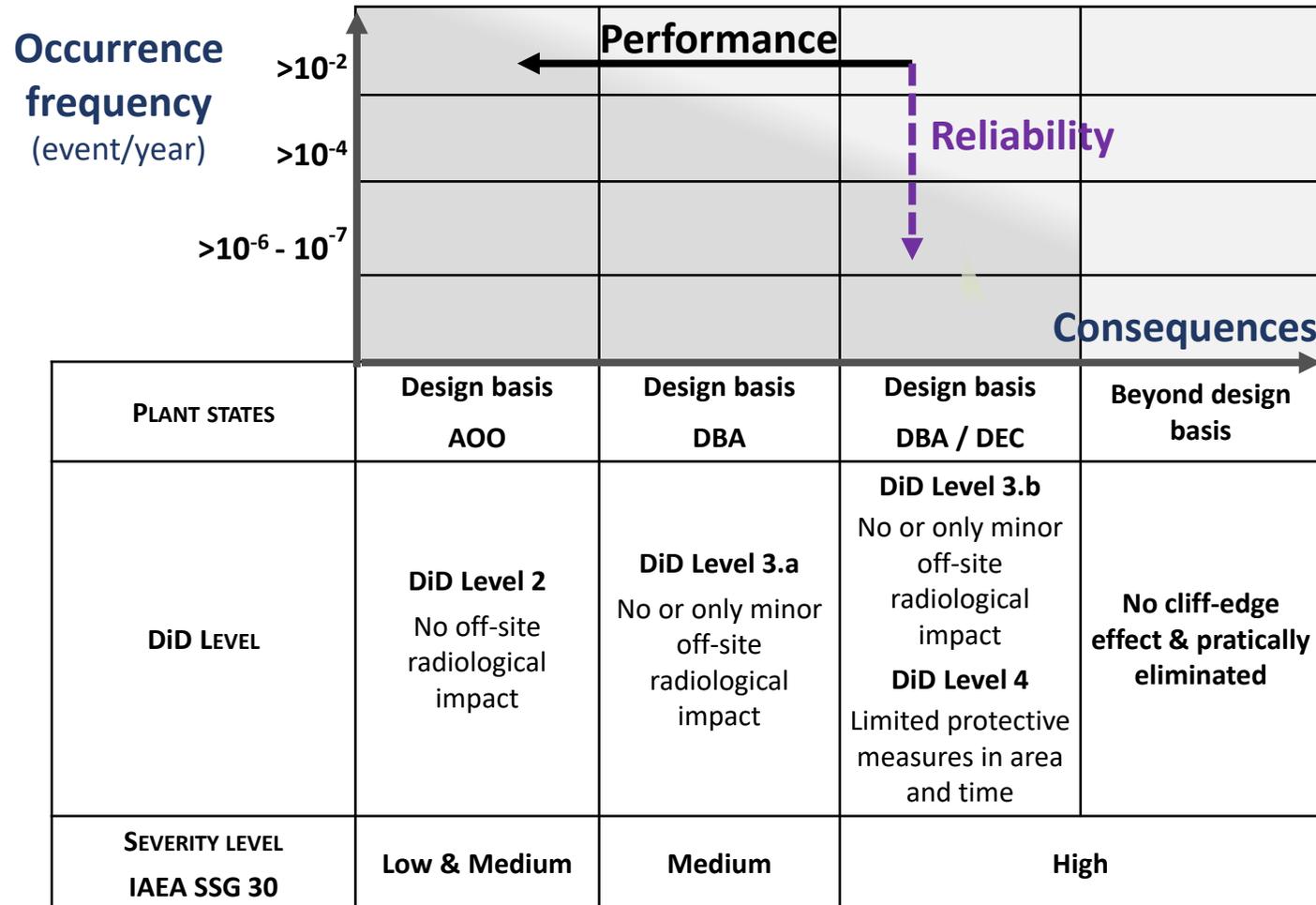


A risk-informed and performance-based design approach to decision-making combines the "risk-informed" and "performance-based" elements.

An approach in which risk insights, engineering analysis and judgment including the principle of defense-in-depth and the incorporation of safety margins, and performance history are used, to

- (1) focus attention on the most important activities,
- (2) establish objective criteria for evaluating performance,
- (3) develop measurable or calculable parameters for monitoring system and licensee performance,
- (4) provide flexibility to determine how to meet the established performance criteria in a way that will encourage and reward improved outcomes, and
- (5) focus on the results as the primary basis for safety decision-making

Risk Domain

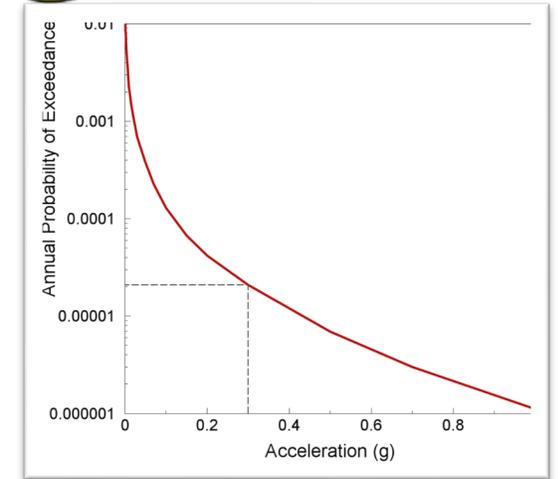
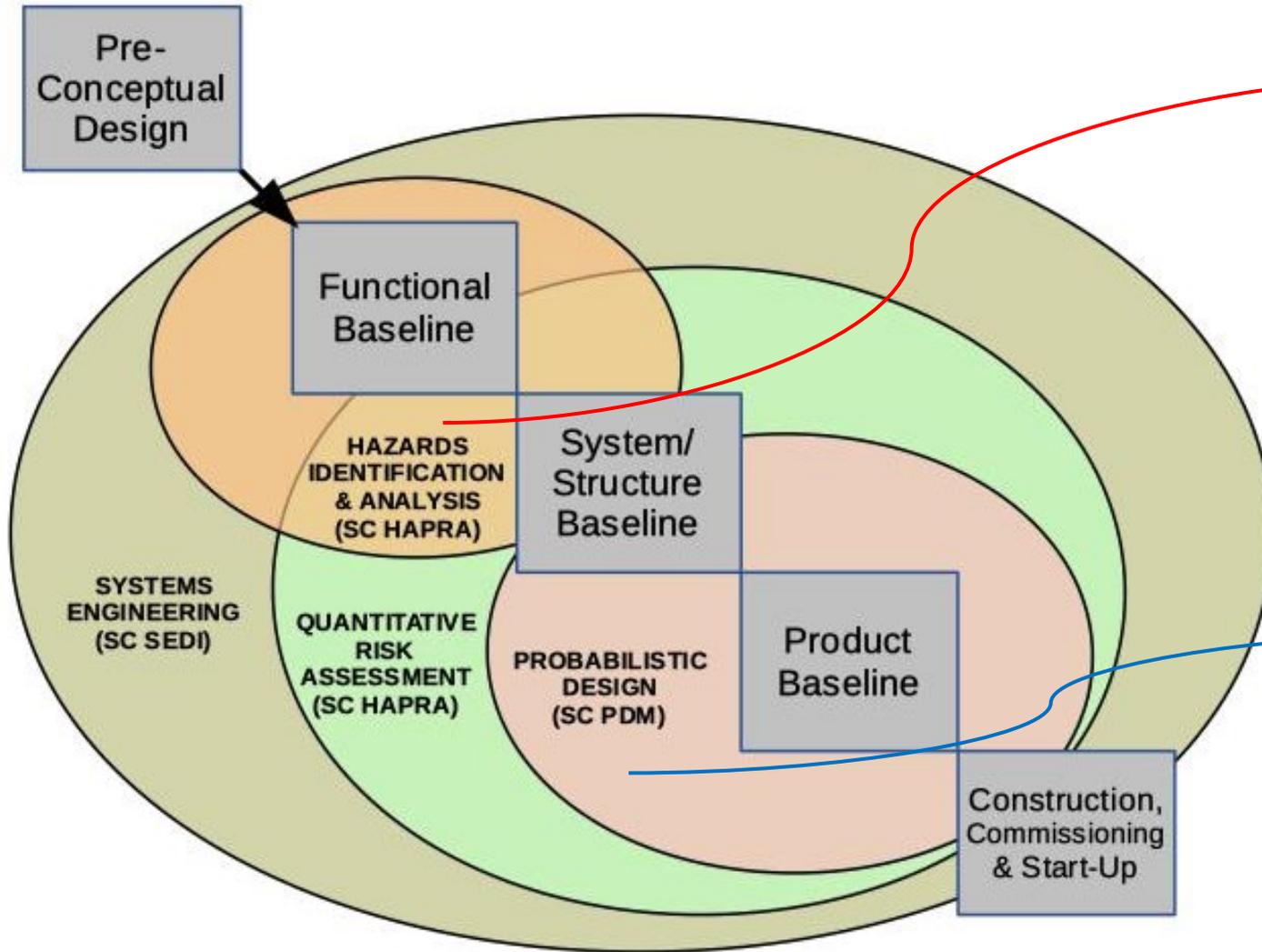


Options to bring an undesirable event that puts a plant into an uncontrolled state back into a controlled or safe state (shaded zone).

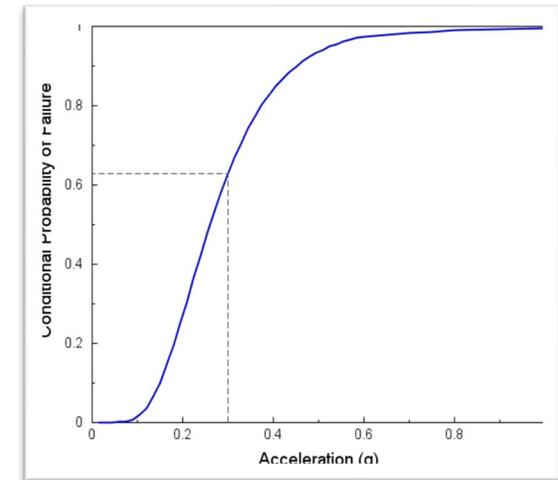
A safety related SSC (more generally, a 'layer of provisions') can be introduced, against an (initiating) event with unacceptable consequence, to :

either reduce the severity of consequence or reduce the frequency of occurrence or both.

PLANT SYSTEMS DESIGN INFLUENCE DIAGRAM r2



Hazard



Fragility

Example: Calculating Risk by Combining Seismic Hazard Curve with Fragility Curve

Hazard curve $H(a)$ and fragility curve $P_f(a)$ can be combined by numerical convolution by either of the two equations:-

$$P_F = - \int_0^{+\infty} P_F(a) \left\{ \frac{dH(a)}{da} \right\} da$$

$$P_F = \int_0^{+\infty} H(a) \left\{ \frac{dP_F(a)}{da} \right\} da$$

Typical hazard curves are close to linear in log-log scale and can be approximated by a power law:

$$H(a) = K_I a^{-K_H}$$

K_I is constant and K_H is slope parameter

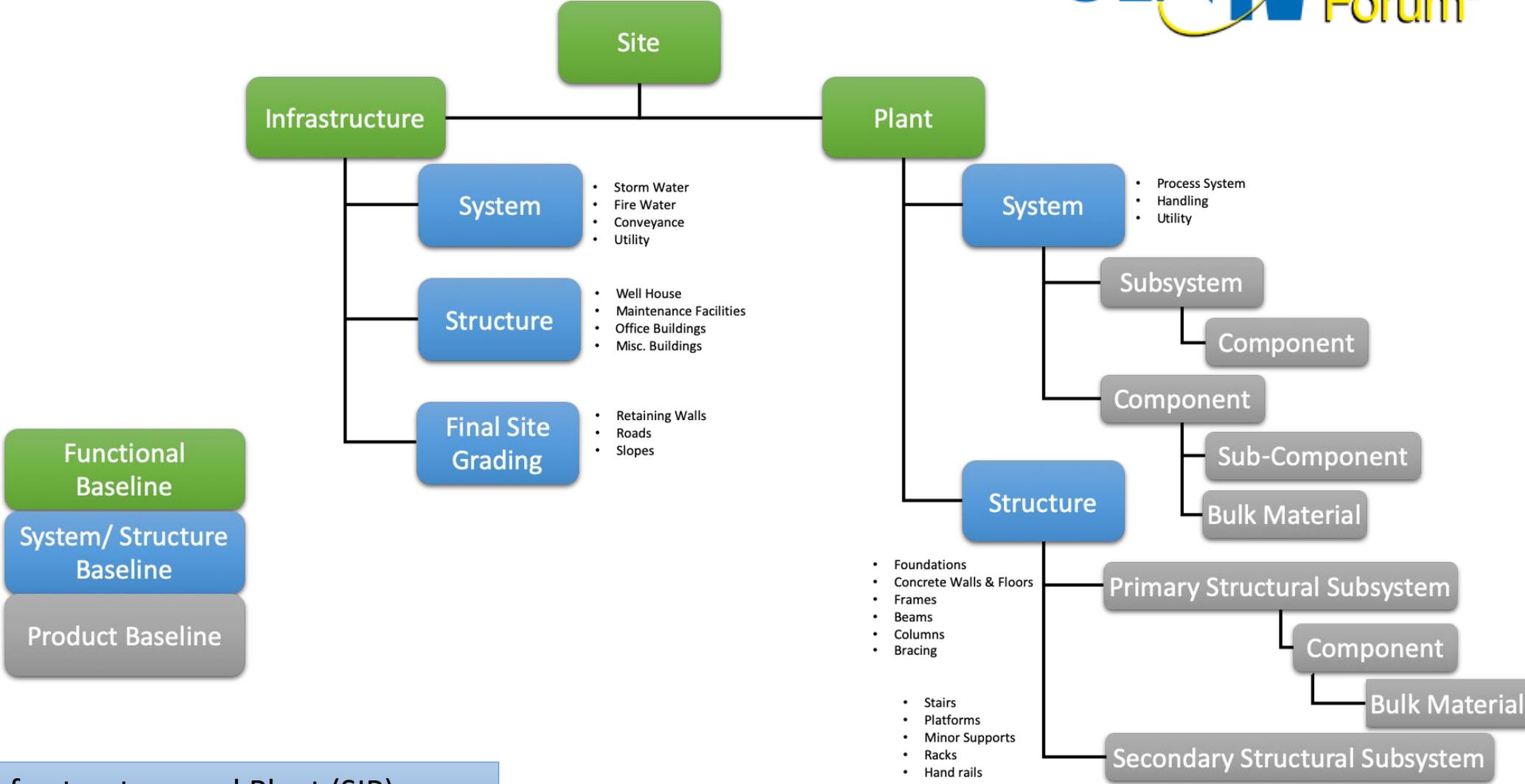
$$P_F = H F_{50\%}^{-K_H} e^{\alpha}$$

$$F_{50\%} = \frac{C_{50\%}}{C_H}$$

$$\alpha = 1/2 (K_H \beta^2)$$

H is reference exceedance frequency, C_H is UHRS ground motion level at H , $C_{50\%}$ is median fragility capacity and β is the logarithmic standard deviation of the fragility.

Plant System Design Taxonomy

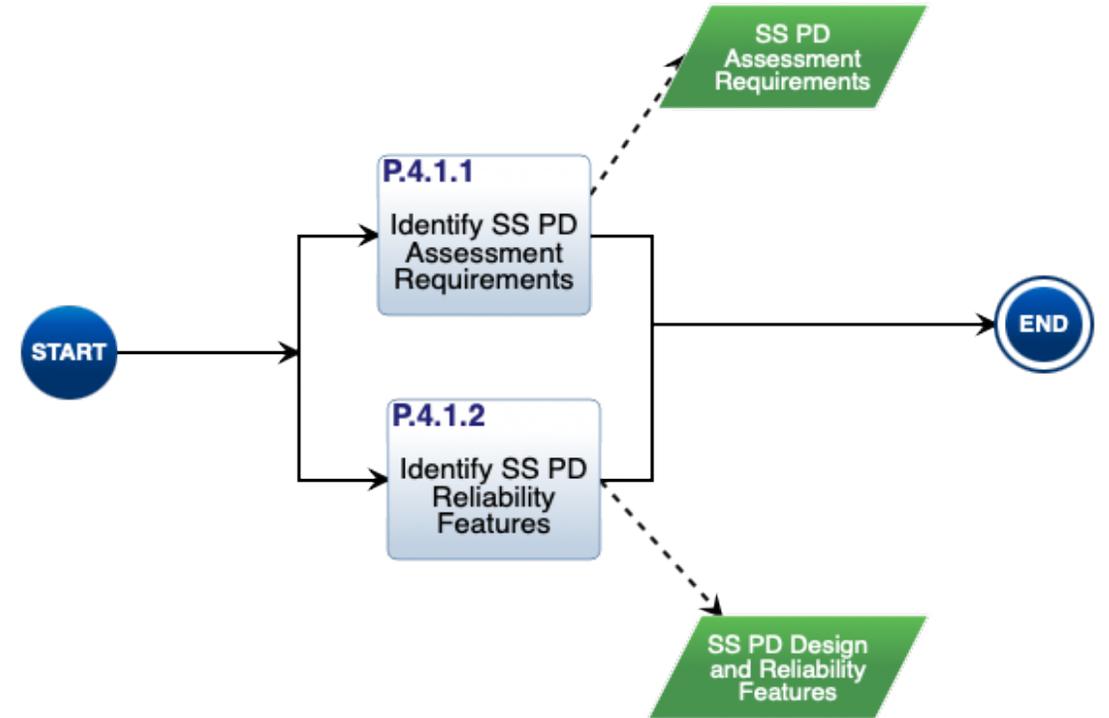


Site, Infrastructure and Plant (SIP)

Project Stage vs Baseline

Project Stages (Example) \ Baselines	Functional	System/Structure	Product
Conceptual	Approved	Preliminary	Preliminary for long-lead and key components
Preliminary	Updates as required	Approved	Preliminary
Final	Updates as required	Updates as required	Approved

PSD Style : Activity Diagrams



P.4.1 is a compilation of
P.4.1.1 and P.4.1.2

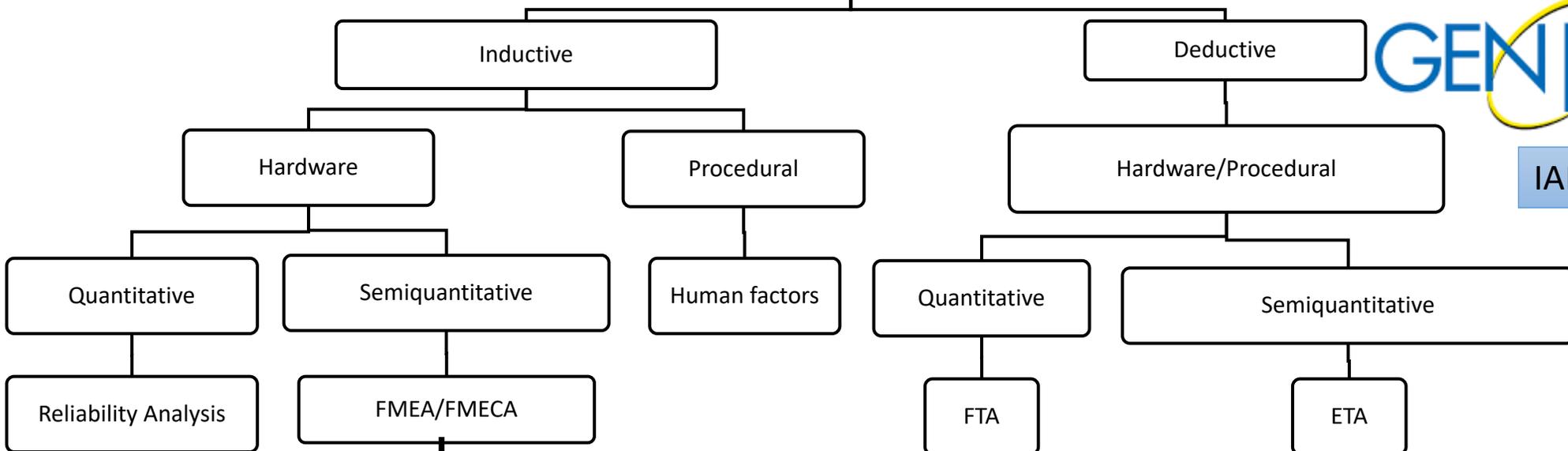
Activity P.4.1 reviews the latest design and reliability information and identifies the key SS systems that contribute the most to the overall plant reliability and availability. The purpose of the activity is to identify which features would benefit the most from a PD approach based on the key requirements from stakeholders.

Table of INPUT – ACTIONS - OUTPUTS



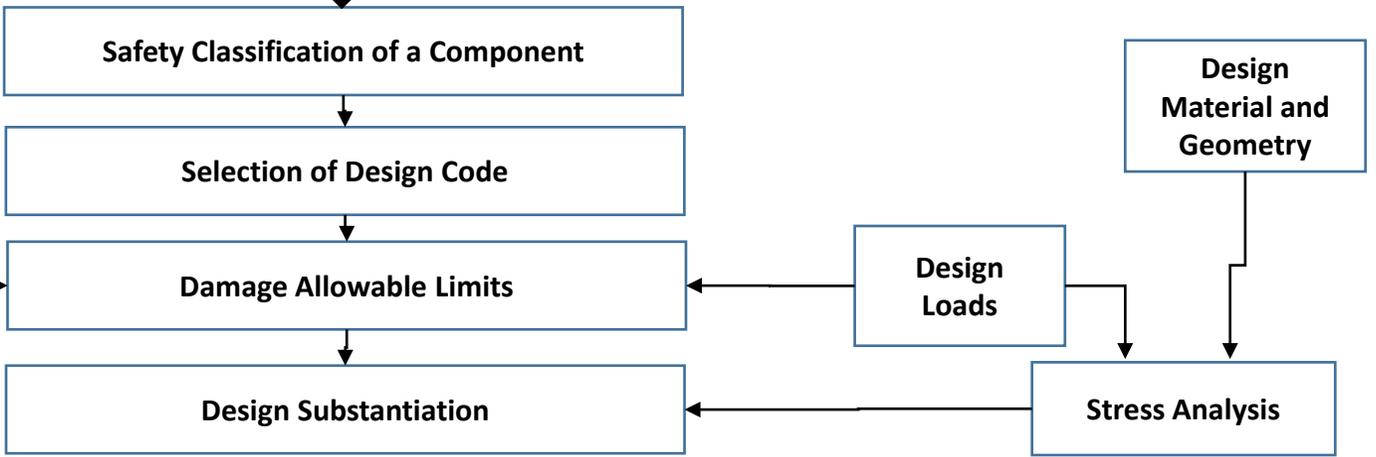
INPUTS	ACTIONS	OUTPUTS
H300 PRE Input to FBL	P.4.1 Review SS Design and Reliability Assessments	P410 SS PD Design and Reliability Features
P300 PDM Input to FBL	P.4.1.1 Identify SS PD Assessment Requirements	P411 SS PD Assessment Requirements
S400 Approved FBL Design Products	P.4.1.2 Identify SS PD Reliability Features	P412 SS PD Design and Reliability Features

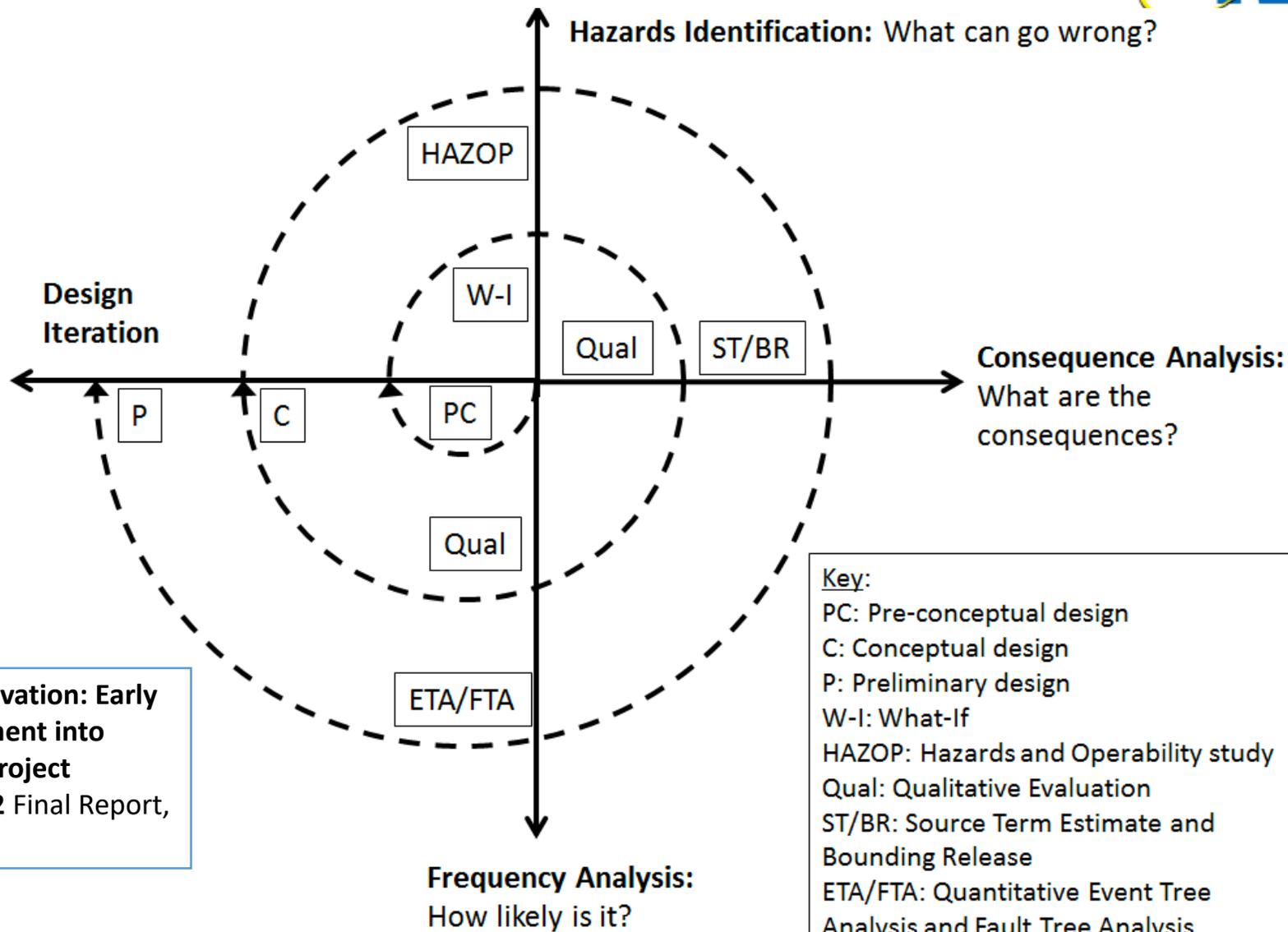
System Safety Analysis



Mode of Failure			Failure Effect				Criticality	Corrective Action	
Mode	Cause	Frequency	Effects	Detection Method	Probability of Detection	Severity	Priority of Risk	Design Modification	Design Verification

- Plastic Collapse
- Buckling
- Fracture
- Fatigue
- Creep
- Leakage
- Corrosion/erosion
- Overturning (overall stability)
- Loss of ductility and strain hardening due to irradiation





Program on Technology Innovation: Early Integration of Safety Assessment into Advanced Reactor Design—Project Capstone Report 3002015752 Final Report, October 2019, EPRI

Timeline for the New PSD Code from ASME



- Approved for publication end of March 2023.
- The timeline for PSD suits well with the Gen IV initiatives.

Conclusions

- Safer but more cost-effective designs needed
- New design approach required in nuclear industry
- Include hazard analysis in early stages of design that
 - advance as the design matures
 - provide structure to the initial development of a probabilistic risk assessment
- Incorporate “Systems Engineering” design processes, practices and tools
- Incorporate risk informed probabilistic design methodologies with traditional deterministic design methods using reliability and availability targets
- Integrate them into the existing design processes and procedures to produce a new ‘Plant System Design’ code

Thank You

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Acknowledgement:

Ralph S. Hill III

Chair, Plant Systems Design Standards Committee, ASME



Special Webinar Event

20th Anniversary Celebration with the participation of current and former GIF Chairmen



Wednesday, April 28, 2021 8:30 am EDT (UTC-4)

Register at: <https://attendee.gotowebinar.com/register/4928218237397954063>

*“Progress and
Future Prospects
toward Deploying
GEN IV Reactors as
Advanced Nuclear
Energy Systems”*

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25 May 2021	Advanced Manufacturing for Gen IV Reactors	Dr. Isabella Van Rooyen, INL, USA
24 June 2021	In Service Inspection and Repair Developments for SFRs and Extension to other Gen4 Systems	Dr. François Baque, CEA, France